

INTRODUCTION OF THE DERIVED LIMITS FOR RADIOACTIVE CONTAMINATION ON SURFACE BASED ON THE ICRP PUBLICATION 61.

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Abstract.

By analyzing the publications of National Radiological Protection Board-NRPB-DL1(1), NRPB-DL2(2) and NRPB-DL2 Supplement(3) that determined the derived limits for surface contamination, based on publications ICRP 26(4) and 30(5), we had determined the new limits based on publications ICRP 60(6) and 61(7) using the same factors found in the publications mentioned above.

1. Introduction.

We had determined the new limits based on publications ICRP 60 and 61 using the same factors found in the publications mentioned above. For this we make a summary of the models and equations adopted in the publications of National Radiological Protection Board-NRPB-DL1, NRPB-DL2 and NRPB-DL2 Supplement and of the modifications that we had to introduce to adjust to the LIAs' values presented in the publication ICRP 61.

2. The Model of Derived Limits for External Irradiation of Hand in Active Areas is:

$$DL = \frac{0,25}{D_1}$$

Where:

DL = derived limit (Bq.cm⁻²)

0,25 is the results of the following considerations:

The skin in active areas, it is assumed that the skin is in continuous contact with the contaminated surfaces during working hours;

The dose equivalent limit for skin recommended by ICRP is 0,5 Sv in a year based on the prevention of non-stochastic effects; For workers exposed for 2000 hours each year.

D₁ = average of the equivalent dose rates per units surface activity at tissue depths of 50 and 100 mm (mSv⁻¹ Bq⁻¹ cm²) (4)

As there was not modified in the ICRP 60 the limit value for hand, the equation still keeps valid, and also the table based on it.

3. The Model of Derived Limits for Inhalation of Resuspended Activity in Active Areas is:

$$DL = \frac{DAC \cdot 10^{-6}}{RF \cdot 10^{-2}}$$

Where:

- DAC = derived air concentration for workers (Bq m⁻³)
 RF = resuspension factor (4)
 10⁻⁶ is a conversion factor of DAC given from m³ to cm³
 10⁻² is a conversion factor of RF given from m⁻¹ to cm⁻¹.

On this equation there is only one variable that changed, the DAC, which assumes values according to ALI's inhalation values from ICRP 61 publication.

4. The Model of Derived Limits for External Irradiation of Hand in the Case of Body Surface Contamination is:

$$DL = \frac{0,057}{D_1}$$

Where:

DL = derived limit (Bq cm⁻²)

0,057 is the results of the following considerations:

The model used for contamination on the skin assumes very cautiously that the contamination does persist throughout the year;

Since the recommended dose equivalent limit for skin is 0,5 Sv in a year;

The average level of skin contamination

should not give a dose equivalent rate to the basal layer of the epidermis exceeding 57 mSv h⁻¹;

The DL is then the surface activity that would deliver this dose equivalent rate to the basal layer.

D₁ = average of the equivalent dose rates per units surface activity at tissue depths of 50 and 100 mm (mSv h⁻¹ Bq⁻¹ cm²) (4)

As there was not modified in the ICRP 60 the limit value for hand, the equation still keeps valid, and also the table based on it.

5. The Model of Derived Limits for Ingestion of Material on the Skin in the Surfaces of the Body is:

$$DL = \frac{ALI}{5 \cdot 50 \cdot 10}$$

Where:

ALI = annual limit of intake via ingestion for workers (Bq).

5 = working days in a week.

50 = weeks in a year.

Contamination on the skin may be transferred to the mouth and swallowed.

It is appropriately cautions to assume that a person ingest all activity from 10 cm² of contaminates skin each working day.

The DL is then activity on 10 cm² of skin, which if ingested each working day, would give an intake corresponding to the ALI for radionuclide of interest.

On this equation there is only one variable that changed, the ALI, which assumes values according to ALI's ingestion values from ICRP 61 publication.

We also did the analysis of the radionuclides that had shown variation in their old standard classification reaching other bands.

6. RESULTS

Table 1- Isotopes Classification

Isotopo Classe		Isotopo Classe		Isotopo Classe		Isotopo Classe		Isotopo Classe	
3 H	VI	64 Cu	IV	111 In	IV	226 Ra	II	238 U	I
14 C	IV	65 Zn	V	123 I	IV	227 Ac	I	239 Pu	I
22 Na	IV	67 Cu	IV	125 I	V	227 Th	II	240 Pu	I
33 P	IV	67 Ga	IV	129 Cs	V	228 Th	I	241 Am	I
35 S	IV	68 Ge	IV	131 Cs	VI	228 Ra	III	241 Pu	III
43 K	IV	68 Ga	IV	147 Sm	III	230 Th	I	242 Pu	I
45 Ca	IV	75 Se	V	147 Pm	IV	231 Pa	I	244 Cm	I
51 Cr	VI	77 Br	V	169 Yb	IV	232 U	I	113m In	IV
52 Fe	III	81 Rb	IV	197 Hg	V	232 Th	I	87m Sr	V
54 Mn	V	85 Sr	V	204 Tl	IV	233 U	I	99m Tc	IV
55 Fe	VI	87 Y	IV	210 Pb	II	234 U	I	DEPL U	II
56 Co	IV	90 Sr	III	210 Po	II	235 U	II	ENR U	II
57 Co	V	90 Y	IV	210 Bi	IV	236 U	II	NAT Th	II
58 Co	IV	99 Mo	IV	223 Ra	II	237 Np	I	NAT U	II
63 Ni	VI	109 Cd	IV	224 Ra	III	238 Pu	I		

Table 2 - Initial Classification of DLs for Active Area Surfaces and Skin

	I	II	III	IV	V	VI
Active Area	0.1	1	10	100	1 000	10 000
Skin	10	10	10	10	100	1 000

7. References

- Harrison, N.T.; Bryant, P.M.; Clarke, R.H.; Morley, F.M. National Radiological Protection Board, NRPB-DL1, The estimation of derived limits. 1979
- Wrixon, A.D.; Linsley, G.S.; Binns, K.C.; White, D.F.; National Radiological Protection Board, NRBP-DL2, Derived limits for surface contamination. 1979
- Wrixon, A.D.; Linsley, G.S.; National Radiological Protection Board, NRBP-DL2 Supplement, Derived limits for surface contamination. 1982
- ICRP Publication 26. Recommendation of the International Commission on Radiological Protection. 1977
- ICRP Publication 30. Limits of intakes of radionuclides by workers. 1979
- ICRP Publication 60. Recommendations of the International Commission on Radiological Protection. 1990
- ICRP Publication 61. Annual limits on intake of radionuclides by workers based on the 1990 recommendations. 1991

Table 3 - DLs for active area surface contamination

ISOTOPE	Exposure pathway			
	external irradiation of skin		Inhalation	
	D1 $mSv.h^{-1}.Bq^{-1}.cm^2$	DL $Bq.cm^{-2}$	DAC $Bq.m^{-3}$	DL $Bq.cm^{-2}$
3H				
14C				
22Na				
33P				
36S				
43K				
45Ca				
51Cr				
54Mn				
52Fe				
56Fe				
56Co				
57Co				
58Co				
63Ni				
64Cu				
67Cu				
65Zn				
67Ga				
68Ga				
68Ge				
75Se				
77Br				
81Rb				
85Sr				
87mSr				
90Sr				
87Y				
90Y				
99mTc				
109Cd				
111In				
113mIn				
123I				
125I				
129Cs				

D1 = average of the equivalent dose rates per units surface activity at tissue depths of 50 and 100 mm ($mSv.h^{-1}.Bq^{-1}.cm^2$)(4)

ISOTOPE	Exposure pathway			
	external irradiation of skin		Inhalation	
	D1 $mSv.h^{-1}.Bq^{-1}.cm^2$	DL $Bq.cm^{-2}$	DAC $Bq.m^{-3}$	DL $Bq.cm^{-2}$
131Cs				
147Pm				
147Sm				
169Yb				
197Hg				
204Tl				
210Pb				
210Bi				
210Po				
223Ra				
224Ra				
226Ra				
228Ra				
227Ac				
227Th				
228Th				
230Th				
232Th				
NATTh				
231Pa				
232U				
233U				
234U				
235U				
238U				
DEPLU				
NATU				
ENRU				
237Np				
238Pu				
239Pu				
240Pu				
241Pu				
242Pu				
241Am				
244Cm				

DAC = derived air concentration for 40h working week ($Bq.m^{-3}$)

Table 4 - DLs for skin contamination

ISOTOPE	Exposure pathway		
	external irradiation of skin		Ingestion
	D1 $mSv.h^{-1}.Bq^{-1}.cm^2$	DL $Bq.cm^{-2}$	ALI Bq
3H			
14C			
22Na			
33P			
35S			
43K			
45Ca			
51Cr			
54Mn			
52Fe			
55Fe			
56Co			
57Co			
58Co			
63Ni			
64Cu			
67Cu			
65Zn			
67Ga			
68Ga			
68Ge			
75Se			
77Br			
81Br			
85Sr			
87mSr			
90Sr			
87Y			
90Y			
99mTc			
109Cd			
111In			
113mIn			
123I			
125I			
129Cs			
131Cs			
147Pm			
147Sm			
169Yb			
197Hg			
204Tl			
210Pb			
210Bi			
210Po			
223Ra			
224Ra			
226Ra			
228Ra			
227Ac			
227Th			
228Th			
230Th			
232Th			
NATTh			
231Pa			
232U			
233U			
234U			
235U			
238U			
238U			
DEPLU			
NATU			
ENRU			
237Np			
238Pu			
239Pu			
240Pu			
241Pu			
242Pu			
241Am			
244Cm			

D₁ = average of the equivalent dose rates per units surface activity at tissue depths of 50 and 100 mm ($mSv.h^{-1}.Bq^{-1}.cm^2$)(4)

ALI = annual limit of intake via ingestion for workers (Bq)

Table 6 - Radionuclides by Class for Skin

I	10	II	10	III	10	IV	10	V	100	VI	1 000
ISOT	Bq.cm-2	ISOT	Bq.cm-2	ISOT	Bq.cm-2	ISOT	Bq.cm-2	ISOT	Bq.cm-2	ISOT	Bq.cm-2
241 Am	2.8E+2	210 Pb	8.0E+2	52 Fe	2.4E+2	210 Bi	2.8E+1	77 Br	3.6E+2	51 Cr	4.2E+3
244 Cm	2.0E+1	210 Po	2.8E+2	90 Sr	1.2E+2	14 C	2.0E+2	57 Co	4.0E+2	131 Cs	4.8E+3
237 Np	1.6E+1	223 Ra	2.8E+2	147 Sm	2.8E+1	45 Ca	6.8E+1	129 Cs	4.6E+2	55 Fe	4.5E+3
238 Pu	1.2E+1	226 Ra	2.8E+2	224 Ra	8.0E+2	109 Cd	3.6E+3	197 Hg	6.7E+2	3 H	4.0E+5
239 Pu	2.8E+2	227 Th	2.8E+2	228 Ra	3.2E+1	56 Co	9.0E+1	125 I	6.4E+2	63 Ni	4.0E+4
240 Pu	2.4E+1	NAT Th	1.2E+2	241 Pu	1.4E+1	54 Mn	1.5E+2	54 Mn	6.6E+2		
242 Pu	3.2E+2	235 U	2.8E+2			64 Cu	5.3E+1	75 Se	6.7E+2		
228 Th	1.2E+2	236 U	3.6E+1			67 Cu	2.6E+1	85 Sr	8.9E+2		
230 Th	1.2E+1	DEPL U	8.0E+0			67 Ga	2.1E+2	87m Sr	9.8E+2		
232 Th	1.6E+1	ENR U	3.6E+1			68 Ga	3.2E+1	65 Zn	1.4E+2		
233 U	1.6E+1	NAT U	8.0E+1			68 Ge	2.5E+1				
234 U	1.6E+1					123 I	1.5E+2				
238 U	1.2E+2					111 In	1.3E+2				
						113m In	6.0E+1				
227 Ac	3.6E+0					43 K	2.6E+1				
231 Pa	4.0E+0					99 Mo	3.4E+1				
232 U	8.0E+1					22 Na	2.9E+1				
						33 P	6.9E+1				
						147 Pm	7.6E+1				
						81 Rb	3.6E+1				
						35 S	1.8E+2				
						99m Tc	2.2E+2				
						204 Tl	2.9E+1				
						87 Y	1.1E+2				
						90 Y	3.2E+1				
						169 Yb	2.5E+1				

ACTIVITIES OF THE INSTITUTE OF RADIATION PROTECTION AND DOSIMETRY ON RADIATION OVEREXPOSURE ANALYSIS

Abstract

Since 1985 the Institute of Radiation Protection and Dosimetry (IRD) has been carrying out a multi-disciplinary group called Radiation Overexposure Analysis Group (ROAG) to study the health effects of occupational radiation overexposure. The group is composed of specialists in radiation protection and dosimetry, epidemiology, toxicology, and medicine. The group's main objective is to estimate the real radiation dose to the worker and, mainly, to make recommendations to correct procedures in occupational situations. The analysis and the follow-up of these cases have been made by the Radiation Overexposure Analysis Group as a multidisciplinary group. It works together with the radiation protection, dosimetry and epidemiology groups. The group was set up to investigate health cases. The group was set up to investigate all cases where the equivalent dose was greater than 100 mSv (twice the annual limit adopted by the Brazilian Commission of Nuclear Energy - CENEA). Presently the group has investigated its performance, procedures and objectives to be more effective.

The personal monitoring laboratories in Brazil are requested to send to the CENEA National Base